



Irradiated Material studies for NSCL and FRIB

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U.S. DEPARTMENT OF
ENERGY

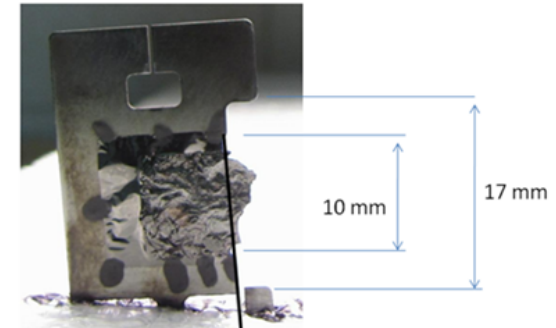
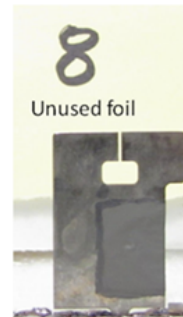
Office of
Science

Outline

- Material study for NSCL charge stripper foil
 - Context
 - Radiation damage studies
- Material study for FRIB beam dump and wedge
 - context and challenges
 - Radiation damage studies
- Future material studies for beam dump and wedge
- Summary

Charge Stripper Studies Context at NSCL

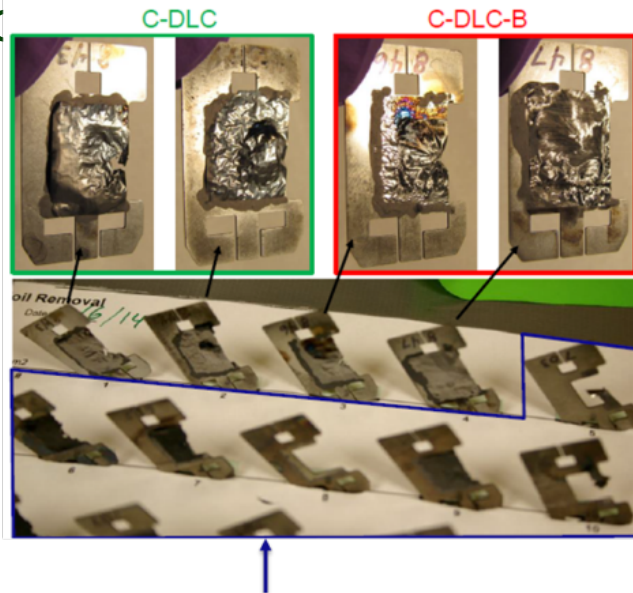
- Charge strippers play an important role in heavy ion beam production facilities
 - Need high charge state to accelerate heavy-ion beam at high energy using small accelerator
 - Carbon stripper foils are one of the dominant types of stripper foils and have been studied for decades
- It is known that thin foils (stripper) used in accelerator suffer a quick degradation due to radiation damage such as swelling and thermo-mechanical changes
 - Limits the lifetime of few hours
- How can we improve the lifetime?
 - Annealing at high temperature
 - Influence of nano-structure on annealing by using multi-layer foil
- SBIR (Small Business Innovation Research) project with NSCL- UHV Technologies, Inc. ,Fort Worth, TX, who developed multi-layers thin foils
 - Award Number: DE-SC0011287, “*Low Z Thin Films for Stripper Foils, Targets and X-Ray Windows*”



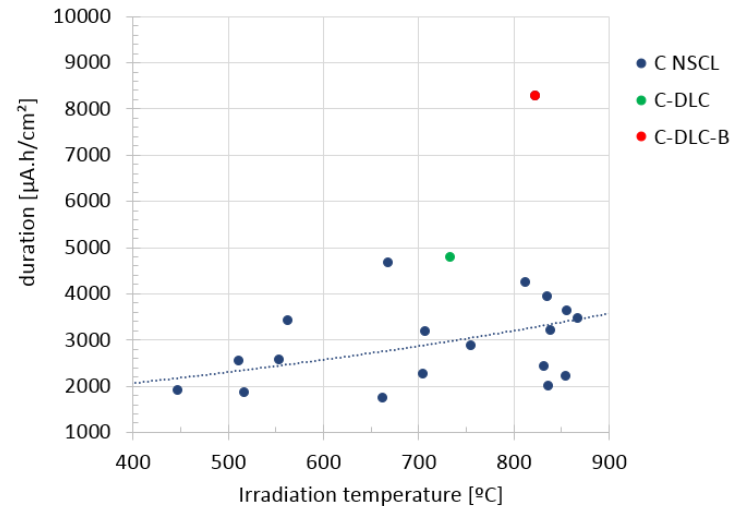
SBIR NSCL/UHV – Phase I

Temperature and nanostructure effect

- 10-multi-layer foils (C-DLC-B and C-DLC) irradiated at NSCL at MSU with $^{78}\text{Kr}^{14+}$ at 13 MeV/u at the stripping injection of the K1200 cyclotron



Current carbon strippers used at NSCL



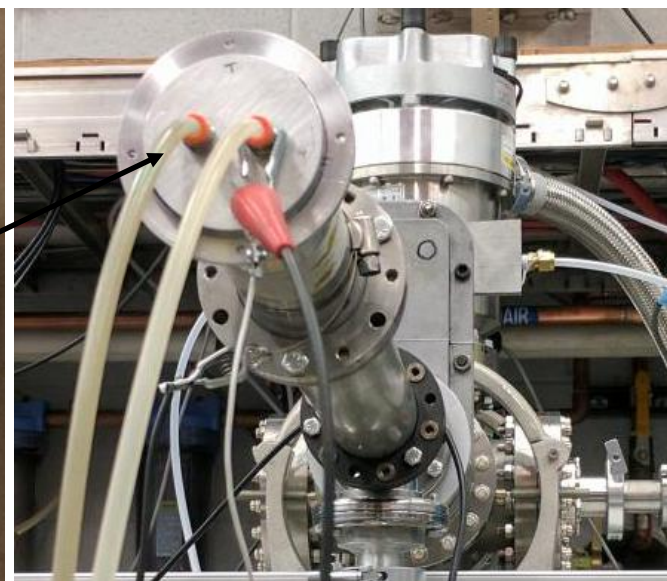
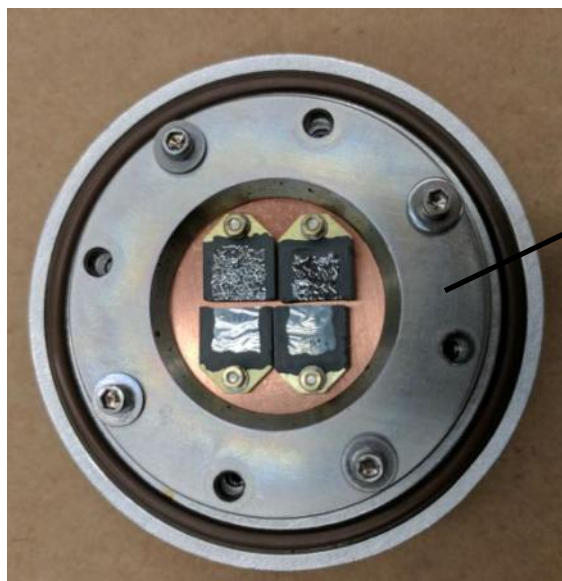
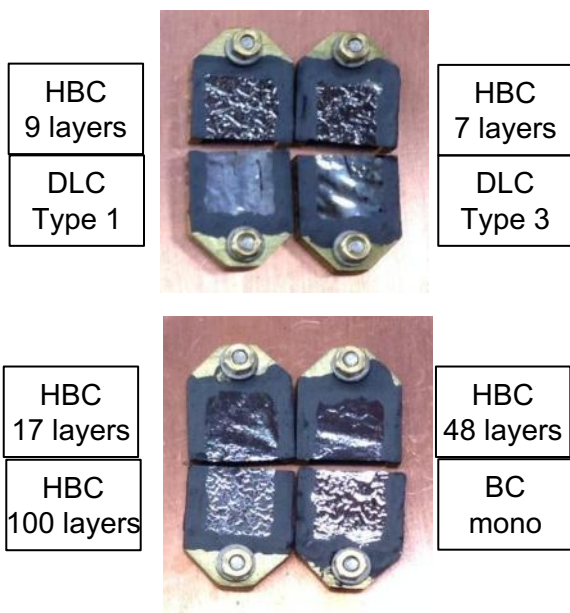
Lifetime time ($\mu\text{A}\cdot\text{h}/\text{cm}^2$) as a function of the irradiation temperature and the microstructure of graphite stripper foils.

- Increased lifetime with irradiation temperature was observed.
- Increased lifetime of the 10 multilayer foils C-DLC-B and C-DLC compare to the standard C-NSCL foils.

SBIR NSCL/UHV – Phase II

Nano-structure effect [1]

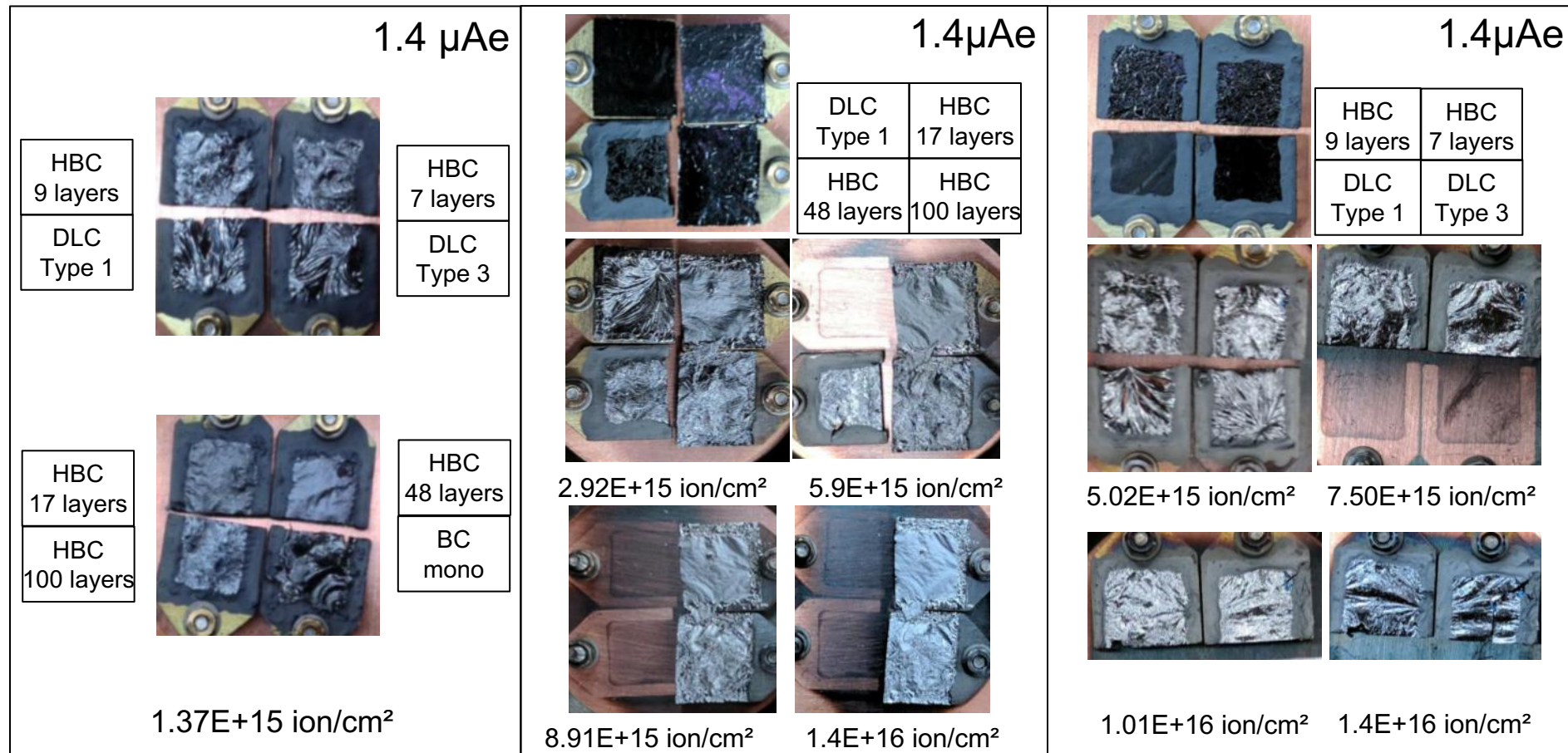
- Ar^{4+} beam at 12MeV using 5U facility at Notre Dame University
- 8 foil types – thickness between 1.7 to 2.5 μm
 - DLC (Diamond Like Carbon): monolayer foil (type 1 and 3)
 - HBC (Hybrid Boron Carbon) : 7, 9, 17, 48, 100 layers
 - BC Monolayer: 10% B + 90% C



SBIR NSCL/UHV – Phase II

Nano-structure effect [2]

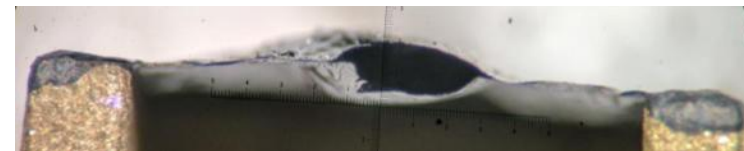
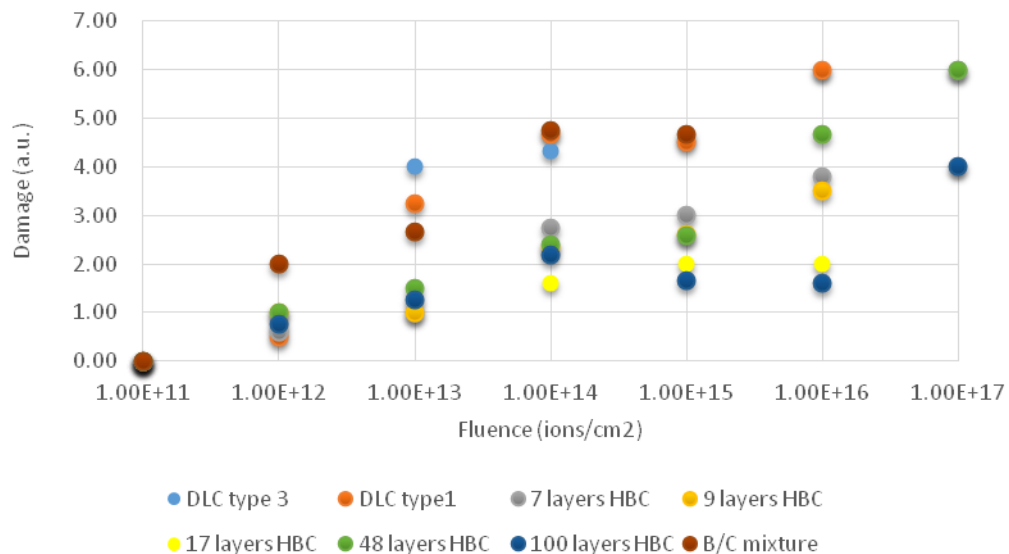
- Increased lifetime of the HBC multilayer foils compared to DLC or BC monolayer



SBIR NSCL/UHV – Phase II

Analyses ongoing

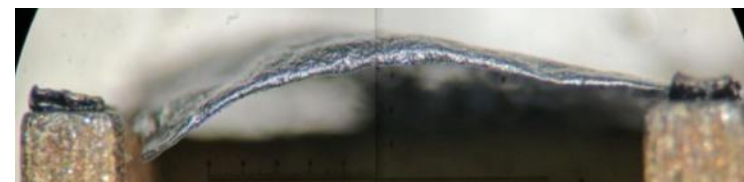
- Better structural stability of 100 layers-HBC foils
 - Nanostructure effect



HBC – 7 layers – 1.4 μAe – 7e14 ions/cm²



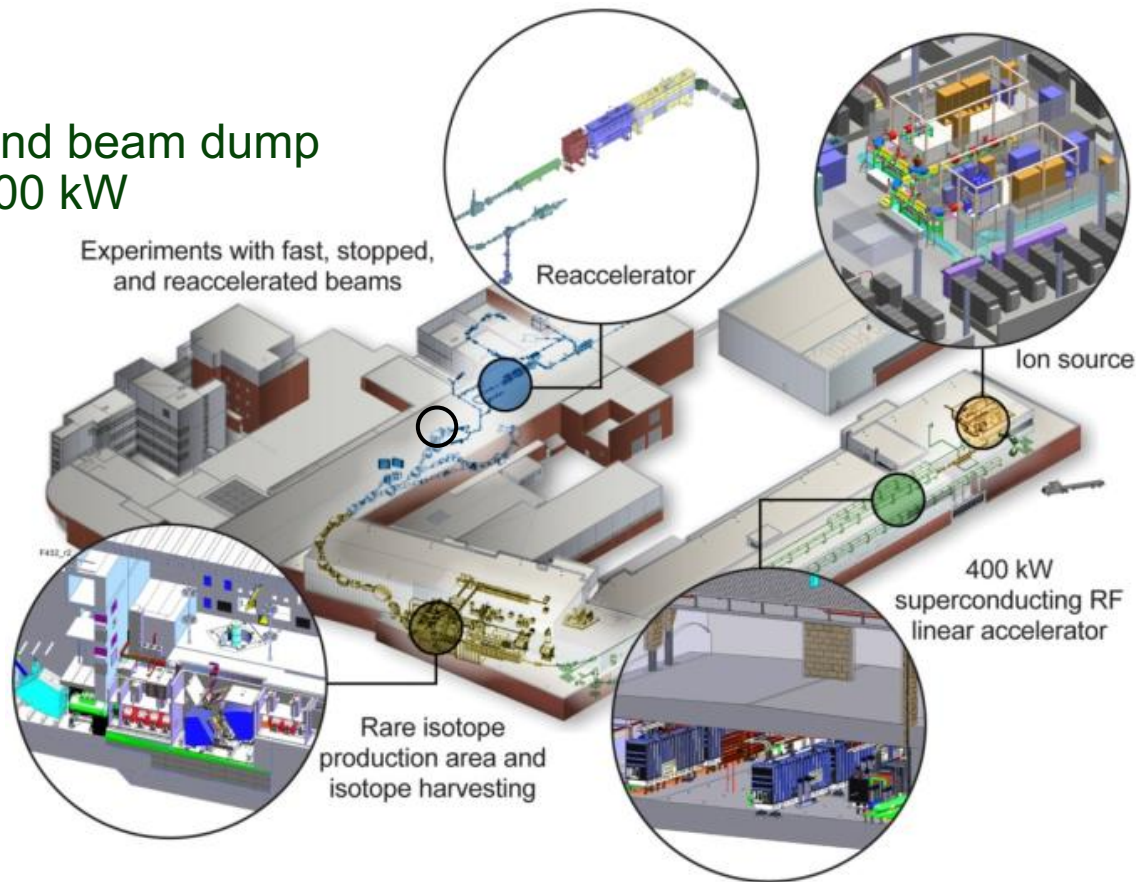
HBC – 100 layers – 1.4 μAe – 7e14 ions/cm²



HBC – 100 layers – 4 μAe – 5e16 ions/cm²

Facility for Rare Isotope Beams

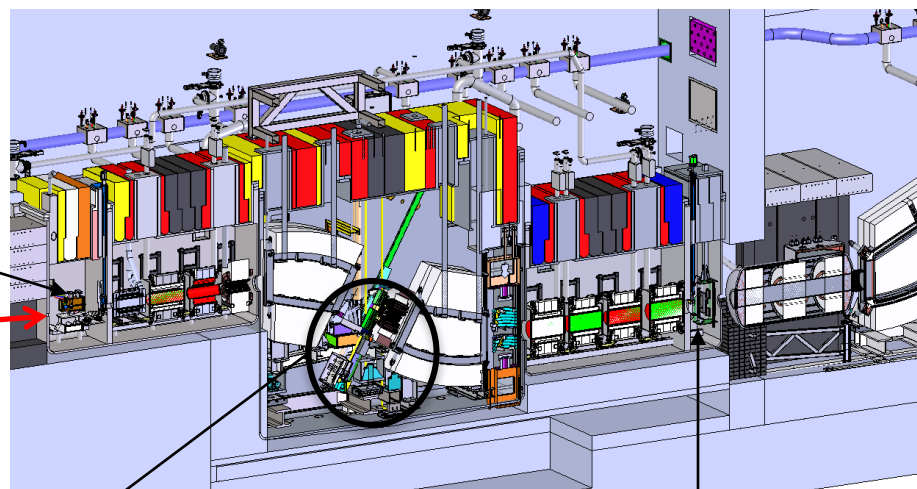
- World-leading heavy ion accelerator facility for rare isotope science
 - Nuclear Structure
 - Nuclear Astrophysics
 - Fundamental Interactions
 - Isotopes for Societal Needs
- Rare isotope production targets and beam dump compatible with beam power of 400 kW at 200 MeV/u for ^{238}U (>200 MeV/u for lighter ions)



Material Challenge in Experimental System Area

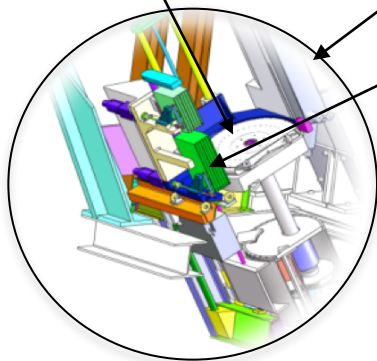
Production target (graphite)

$E = 202 - 260 \text{ MeV/u}$
 $P_{\text{beam}} = 400 \text{ kW}$
 $P_{\text{deposited}} \sim 100 \text{ kW}$
 $\sigma_{x \text{ beam}} = 0.24 \text{ mm}$
 $\sigma_{y \text{ beam}} = 0.29 \text{ mm}$
 $P = 60 \text{ MW/cm}^3$
Dose $\sim 8 \text{ dpa}$



Beam dump drum (Ti-alloy)

$E = 156 - 260 \text{ MeV/u}$
 $P_{\text{deposited}} \sim 325 \text{ kW}$
 $\sigma_{x \text{ beam}} = 1-10 \text{ mm}$
 $\sigma_{y \text{ beam}} = 2-50 \text{ mm}$
 $P = 30 \text{ MW/cm}^3$
Dose $\sim 7 \text{ dpa}$



Fragment catcher (Al-alloy)

$E = 156 - 260 \text{ MeV/u}$
 $P_{\text{deposited}} < 10 \text{ kW}$
 $\sigma_{x \text{ beam}} = \text{up to } 15 \text{ cm}$
 $\sigma_{y \text{ beam}} = \text{up to } 5 \text{ cm}$
 $P = 1.2 \text{ kW/cm}^3$
Dose $\sim 2.5 \text{ dpa}$

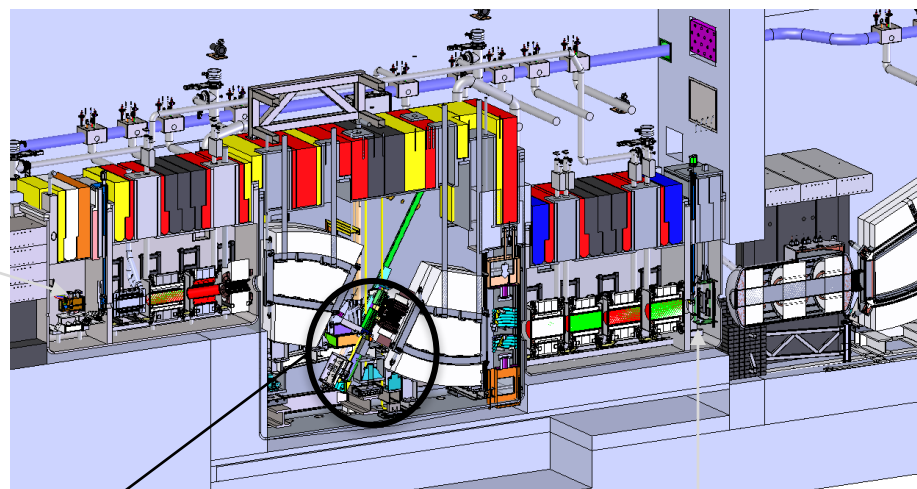
Wedge (Al-alloy)

$E = 156 - 260 \text{ MeV/u}$
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 $\sigma_{\text{beam}} = \text{mm-cm}$
 $P = 13 \text{ kW/cm}^3$
Dose $\sim 1 \text{ dpa}$

Material Challenge in Experimental System Area

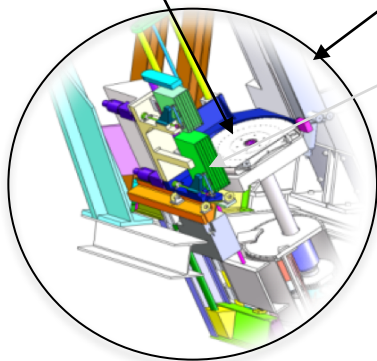
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 $P = \text{W/cm}^3$
Dose $\sim 2.5 \text{ dpa}$

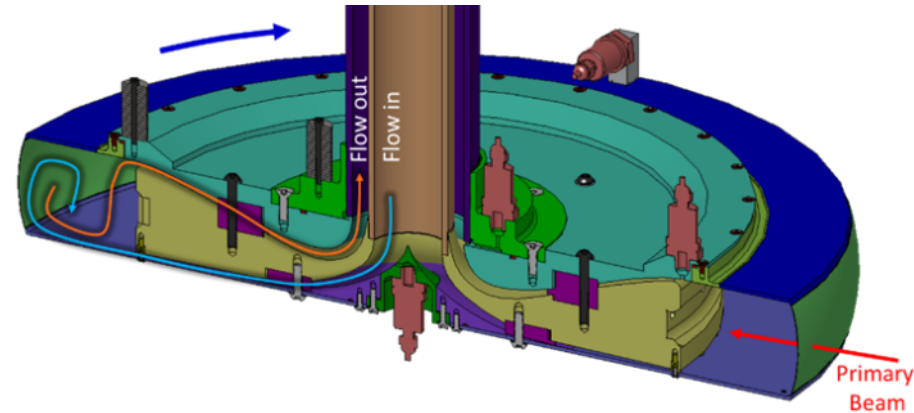
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Dose $\sim 1 \text{ dpa}$

Primary Beam Dump

Water-filled Rotating Drum Concept

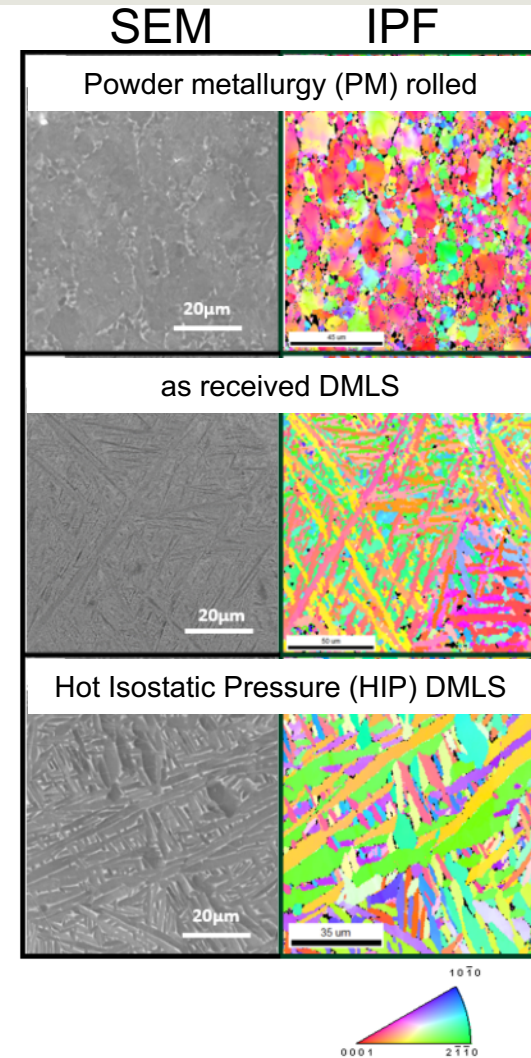
- Beam Dump requirements
 - High power capability up to 325 kW
 - 1 year (5500 h) lifetime desirable
 - » dpa (U beam) ~ 7 ($\sim 4 \cdot 10^{-7}$ dpa/s)
 - Remote replacement and maintenance
- Water-filled rotating drum concept chosen for FRIB baseline
 - Using water to stop the primary beam and absorb beam power
 - Shell temperature limited to 150°C to avoid nucleate boiling of cooling water
- Design parameters
 - Ti-alloy shell thickness 0.5 mm to minimize power deposition in shell
 - 600 rpm and 70 cm diameter to limit maximum temperature and amplitude of temperature changes
 - 60 gpm water flow to provide cooling and gas bubble removal
 - 8 bar pressure inside the drum increases water boiling point to 150°C
- Ti-6Al-4V was chosen as candidate material for the beam dump shell



3D Printed Material for Beam Dump Drum

Which impact on material properties/behavior?

- 3D printing (with Direct Metal Laser Sintered DMLS process) likely fabrication process for FRIB beam dump shell
 - Microstructure of Ti-6Al-4V alloys may change due to fabrication process
 - » Characterization of different microstructures (Powder Metallurgy (PM) rolled, 3D printed (As received DMLS), 3D printed Hot Isostatic Pressure (HIP) (HIP-ed DMLS) ongoing
 - » 3D printed sample analysis shows lamellar microstructure
- The HIP (Hot Isostatic Pressure) was performed at a temperature of 900°C at 102 MPa for 2 hours
- The powder used for DMLS samples is Grade 23 Ti-6Al-4V



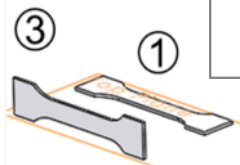
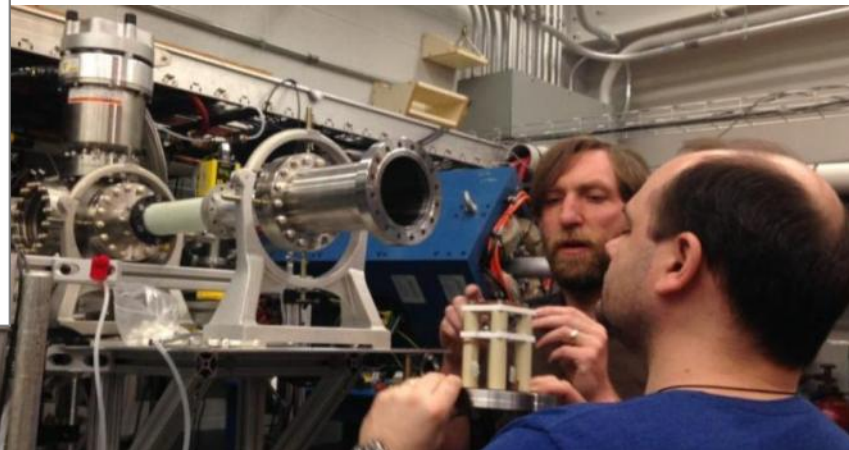
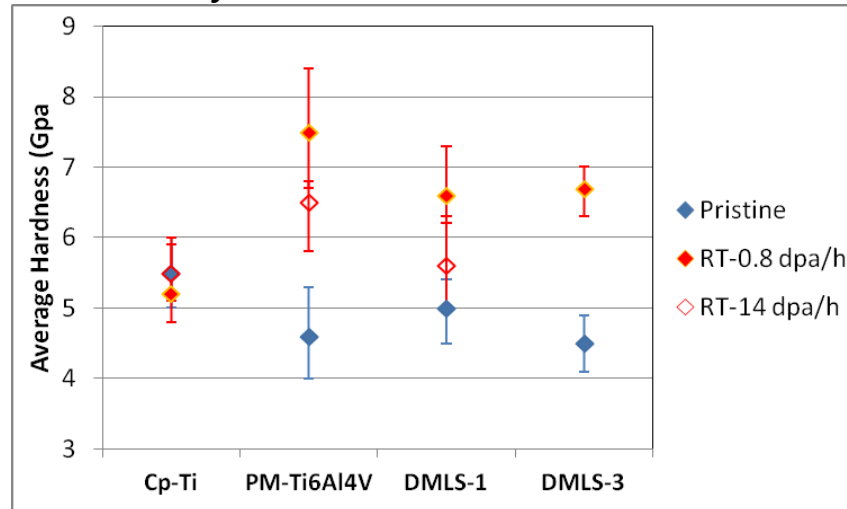
3D Printed Material for Beam Dump Drum

Build Orientation and Radiation Damage Effect [1]

- Irradiation at University Notre Dame with $^{40}\text{Ar}^{2+}$ at 4 MeV up to a fluence of $5 \cdot 10^{16}$ ions/cm² (14 dpa)
 - 4 different microstructures of Ti-alloy: Cp-Ti, PM-Ti6Al4V, DMLS-1, DMLS-3
 - at room temperature and at 350°C
 - Two different dose rates (~ 14 dpa/h and 0.8 dpa/h)



Preliminary results



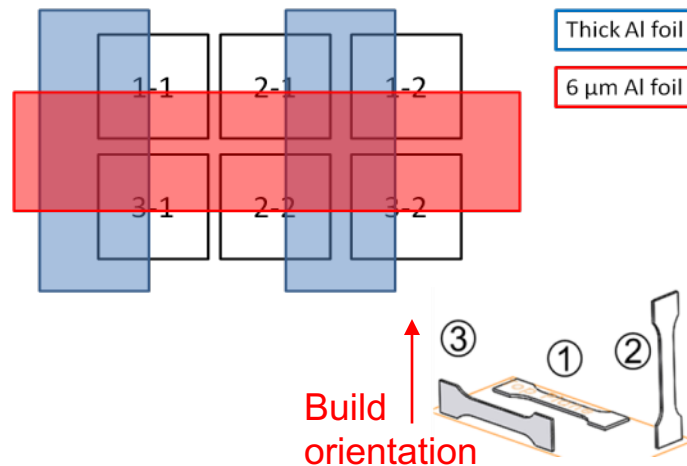
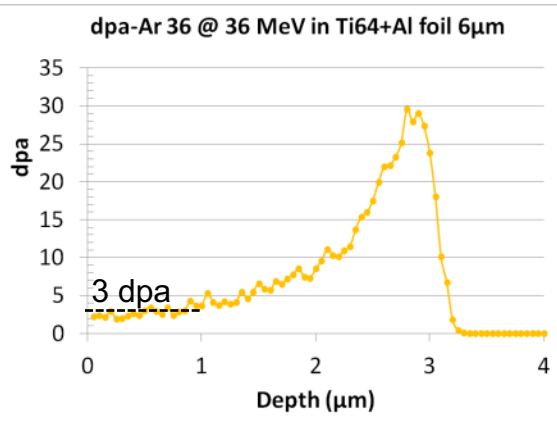
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Build Orientation and Radiation Damage Effect [2]

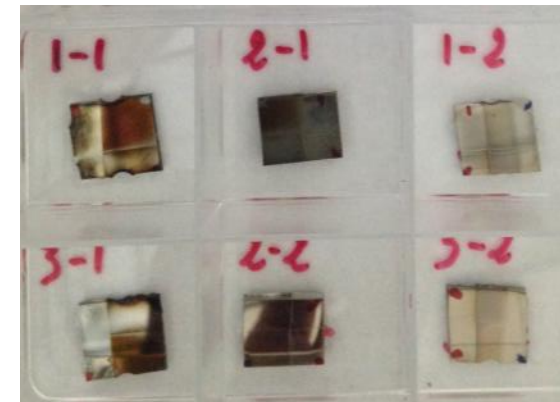
CiMap

- Low energy $^{36}\text{Ar}^{10+}$ beam irradiation performed in July (18-24)
- 3 different doses / sample by using a 6 μm and thick Al foil

Ref	Temp	Surface [cm ²]	Fluence [ions/cm ²]	time	dpa average in 1 microns	dpa average in 2 microns	dpa average in 1 microns after Al foil	dpa average in 2 microns after Al foil	dpa Bragg Peak
Ti64-1-1	150	7.5	2.38E+16	16h+26h	0.644	0.773	2.818	4.392	29
Ti64-1-2	150	12	6.00E+15	16h	0.163	0.195	0.712	1.11	7.1
Ti64-2-1	150	7.5	2.38E+16	16h+26h	0.644	0.773	2.818	4.392	29
Ti64-2-2	150	12	6.00E+15	16h	0.163	0.195	0.712	1.11	7.1
Ti64-2-3	RT	9	3.74E+15	21h	0.101	0.122	0.443	0.691	4.5
Ti64-3-1	150	7.5	2.38E+16	16h+26h	0.644	0.773	2.818	4.392	29
Ti64-3-2	150	12	6.00E+15	16h	0.163	0.195	0.712	1.11	7.1



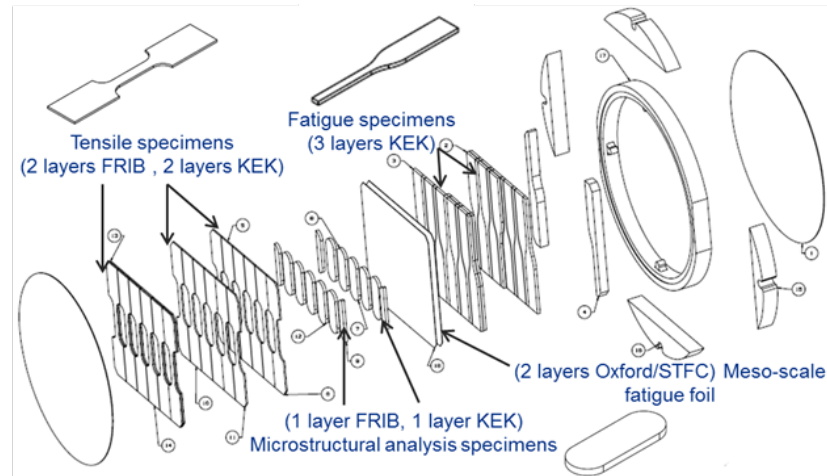
$T_{\text{irr}} = 150^\circ\text{C}$



3D Printed Material for Beam Dump Drum

Build Orientation and Radiation Damage Effect [3]

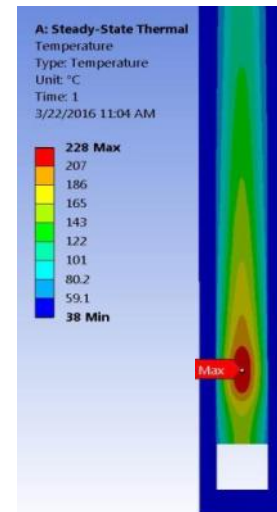
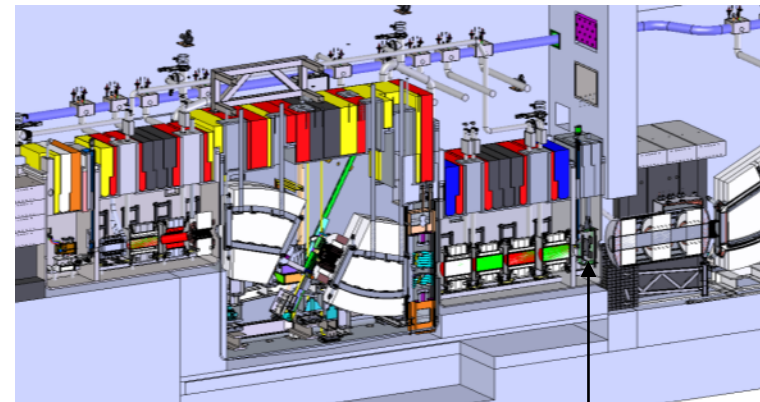
- Test at BNL-BLIP facility started in April 2017 with RaDIATE collaborators
 - Irradiate several Ti-alloys with high energy protons for an extended period of time (8 weeks, up to 1 dpa in Ti samples) and characterize property changes due to proton induced damage
 - FRIB and KEK will irradiate DMLS and conventional Ti-alloy (Grade 5 and 23), compare and share results



Facility for Rare Isotope Beams
U.S. Department of Energy Office of Science
Michigan State University

Wedge context [1]

- Wedge needed to compress momentum spread of isotope beams in high resolution Fragment Separator
 - Each fragment beam likely requires a unique wedge (slope, thickness)
 - High power density needs to be dissipated
- Material studies to assess lifetimes
 - High temperature Aluminum alloy is necessary to support high temperature and high stress due to thermo-mechanical constraint



Wedge (Al-alloy)

$E = 156 - 260 \text{ MeV/u}$

$P_{\text{deposited}} < 2 \text{ kW}$

$\sigma_{\text{beam}} = \text{mm-cm}$

$P = 13 \text{ kW/cm}^3$

Dose $\sim 1 \text{ dpa}$

Wedge width = 10mm

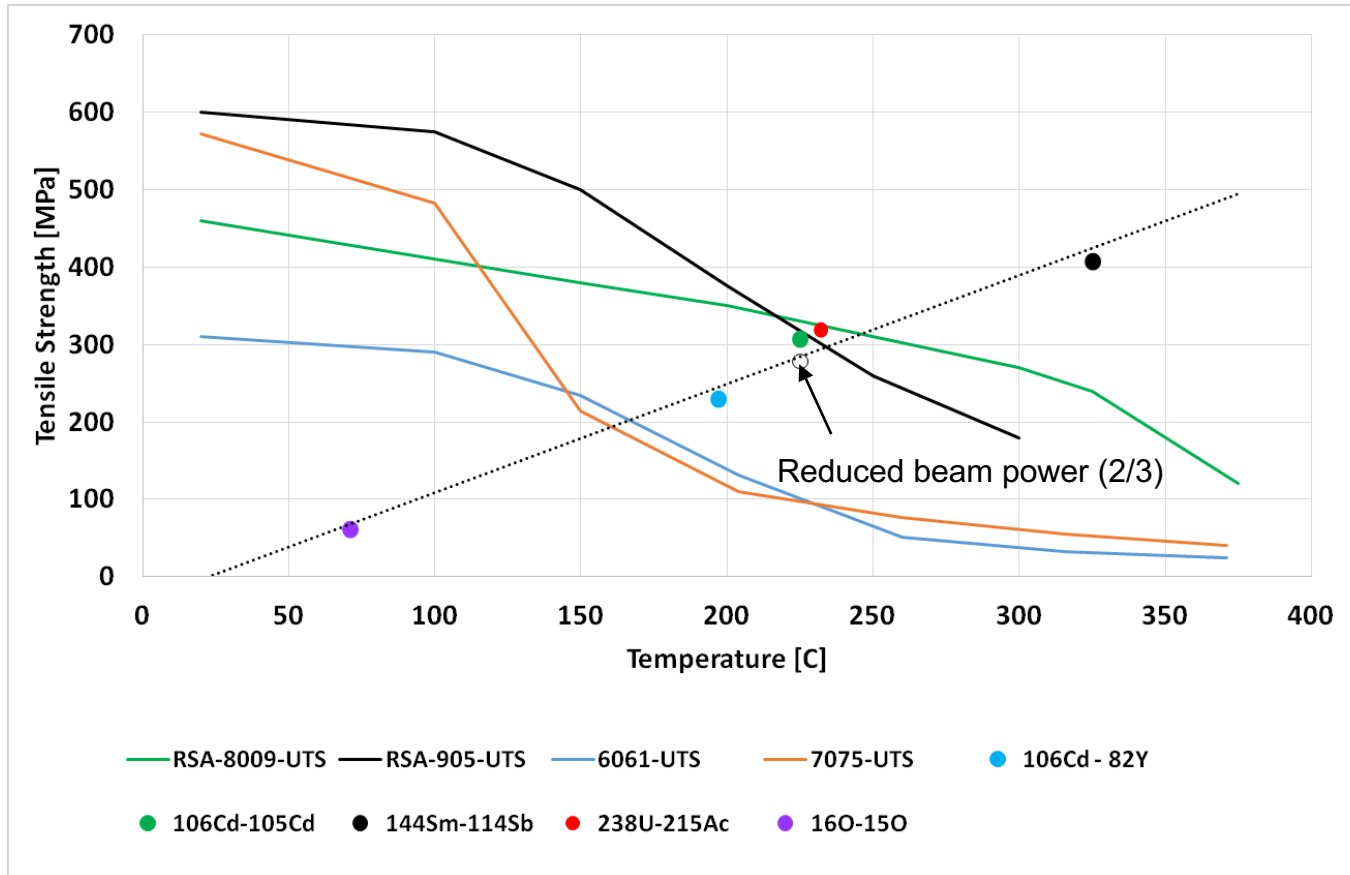
$L = 25 \text{ cm}$

$t = 0.9 - 1.9 \text{ mm}$

Wedge $^{144}\text{Sm} - ^{114}\text{Sb}$
settings 10 mm

Wedge context [2]

- 3 alloys are under consideration
 - 6061, 7075 and RSA 905



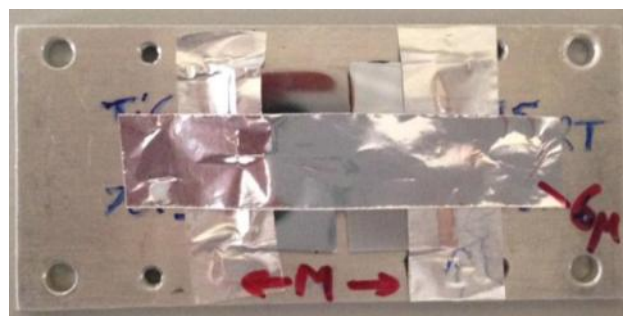
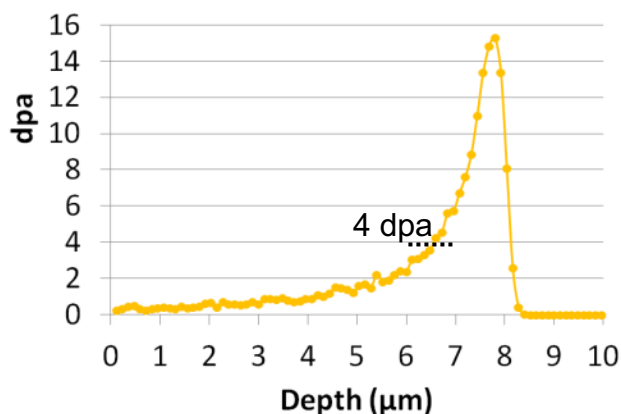
Al-alloy study for Wedge Radiation Damage Effect

CiMap

- Low energy $^{36}\text{Ar}^{10+}$ beam irradiation performed in July (18-24)
- 3 different doses / sample by using a 6 μm and thick Al foil

Ref	Temp	Surface [cm ²]	Fluence [ions/cm ²]	time	dpa average in 1 microns	dpa average in 2 microns	dpa average in 1 microns after Al foil	dpa average in 2 microns after Al foil	dpa Bragg Peak
Al-6061-1	RT	9	3.74E+15	21h	0.083	0.095	1.007	1.784	3.6
Al-6061-2	180-200	6.6	1.56E+16	18h	0.347	0.397	4.216	7.471	15
Al-6061-3	180-200	6.6	1.56E+16	18h	0.347	0.397	4.216	7.471	15
Al-7075-1	RT	9	3.74E+15	21h	0.083	0.095	1.007	1.784	3.6
Al-7075-2	180-200	6.6	1.56E+16	18h	0.347	0.397	4.216	7.471	15
RSA905-1	RT	9	3.74E+15	21h	0.083	0.095	1.007	1.784	3.6
RSA905-2	180-200	6.6	1.56E+16	18h	0.347	0.397	4.216	7.471	15

dpa-Ar 36 @ 36 MeV in Al



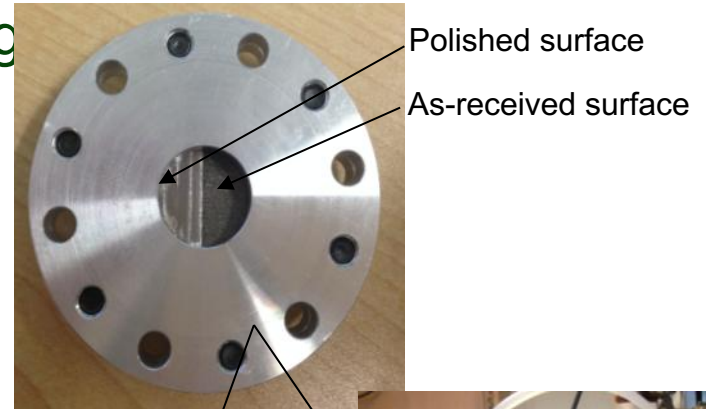
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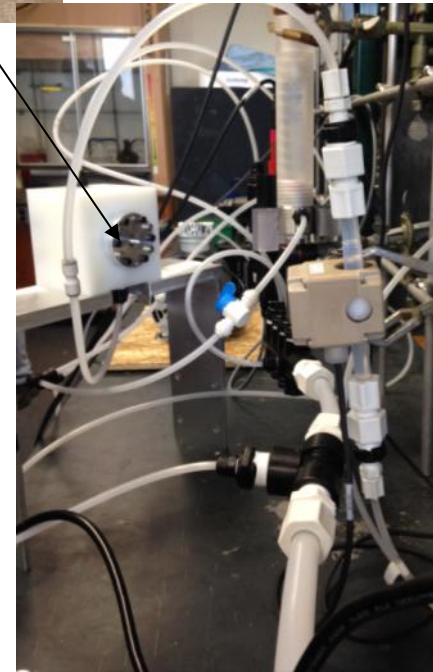
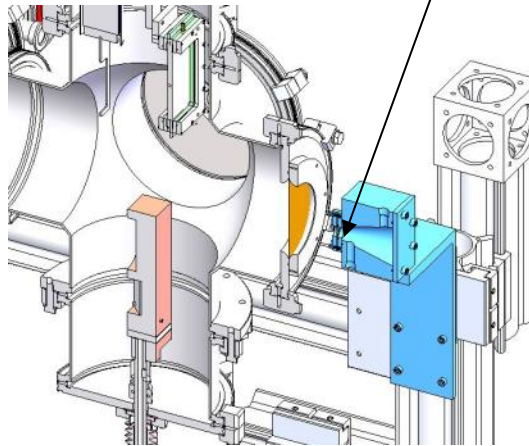
Material studies Path Forward

■ Corrosion Test for a Better Understanding of Lifetime of Beam dump shell

- Short irradiation at NSCL with high energy heavy ion beam
 - » Windows made of 3D printed Ti-6Al-4V
 - » Water cell fabricated
 - » Benchmark gas production in water
 - » Study first stage of corrosion on Ti-6Al-4V



■ Analyses of low energy irradiation samples to investigate mechanical property changes



Conclusion

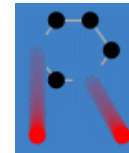
- Graphite stripper gain in lifetime when using nanostructure with multi-layer foils
- Beam Dump and wedge face various thermo-mechanical and chemical challenges
 - Some effects may be enhanced by the presence of the other
 - » corrosion in presence of radiation, stress limit change in the presence of radiation
- Up to now no facility exists to study the impact from all effects combined together
 - The changes of material have to be studied case-by-case experimentally
- 3D printed Ti-6Al-4V seems to be better radiation resistance than standard Ti-6Al-4V
 - Less mechanical property change observed from several studies promises good radiation resistance of this alloy and gives confidence in lifetime
- Investigation on Al-alloys started to support FRIB wedge design

Acknowledgements

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 - F. Durantel, C. Grygiel, I. Monnet, M. Toulemonde
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- UHV Technologies, Inc
 - N. Kumar
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 - A. Amroussia, S. Balachandran Nair, C. Boehlert, A. Lee
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 - M. Avilov, W. Mittig, B. Peruski, B. Phillips, G. Severin, J. Wen

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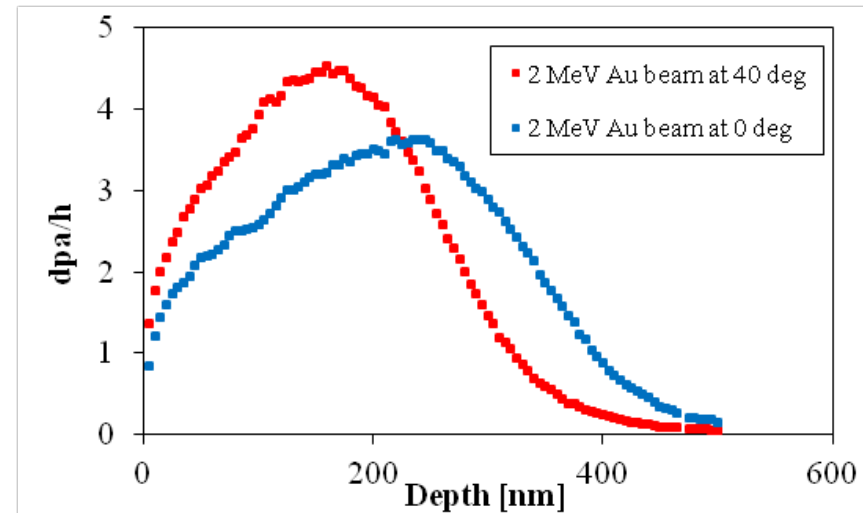
SBIR with UHV Technologies, Inc., Fort Worth, TX. Award Number: DE-SC0011287. DE-FOA-0001193, SBIR/STTR FY 2015 Phase II Release 1



Facility for Rare Isotope Beams
U.S. Department of Energy Office of Science
Michigan State University

Need expert on TEM for Ti-6Al-4V

- In-situ TEM on Ti-alloy at JaNNUS-Orsay (France) accepted next year between February and Spring 2018 (to be defined)
- Our TEM expert left
- Instead of canceling the experiment we would like to share it if somebody is interested in TEM study focussed on Ti-alloy
 - Possibility to add one or 2 samples of other material
- JaNNUS : ARAMIS + TEM
 - 5 days
 - Au beam at 2 MeV
 - Minimum flux = $1\text{e}11$ ions/cm².s



Flux = 10^{11} ions.cm⁻².s⁻¹

Thank you for your attention

